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## MEASUREMENT OF THE Np-237 NEUTRON CAPTURE CROSS SECTION FROM THERMAL TO 100 eV

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For a study of the incineration of nuclear waste, the transmutation of Np-237 has a first priority due to its amount and activity in a burn-up fuel. The existing limited experimental data on capture cross section of Np-237 in resolved resonance range below  $\sim 200$  eV have been obtained using small efficiency g-ray detectors with or without pulse-weighting technique. Therefore there is obvious necessity in new measurements of neutron capture cross section of Np-237 utilizing another detection technique to improve experimental database for this nucleus.

The present time-of-flight measurements were carried out using the 46 MeV electron Linac of KURRI as a pulsed neutron source and the large 8.54 liter 16-section BGO scintillation detector of JNC as a total energy detector of prompt capture  $\gamma$ -rays. A newly developed data taking system based on a 40 MHz FLASH ADC enabled to perform the measurements with a highly radioactive 1.1 g sample of Np-237. The capture cross section of Np-237 was measured relative to standard neutron absorption cross section of B-10. Absolute normalization of the capture cross section was carried out using a thermal value obtained from the analysis of Np-237 total cross section measured with the same sample below 1 eV.

The results of neutron capture cross section measurements in the energy range from thermal to 100 eV and total cross section measurements from thermal to 1 eV are presented.